Deuterium permeation in tungsten by mixed ion irradiation

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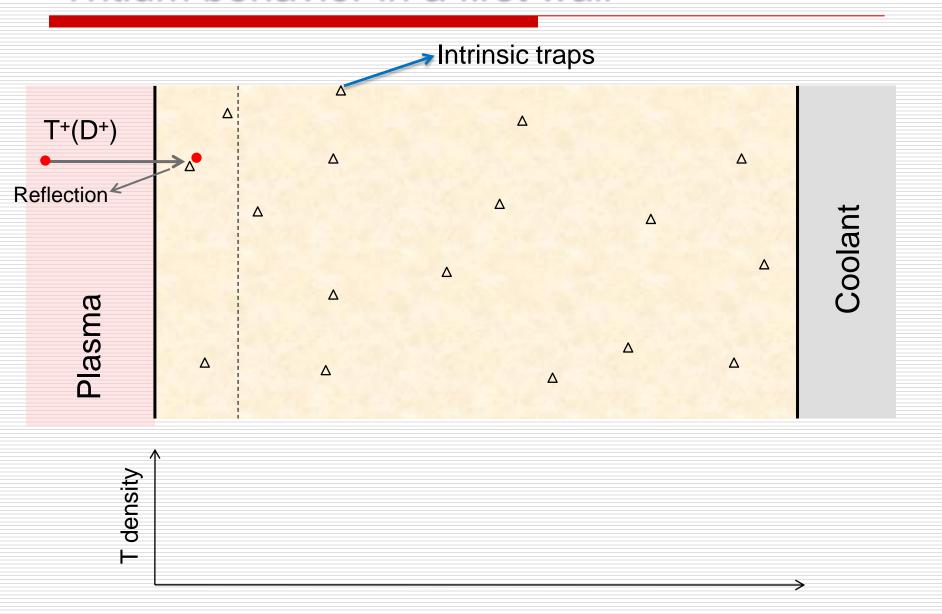
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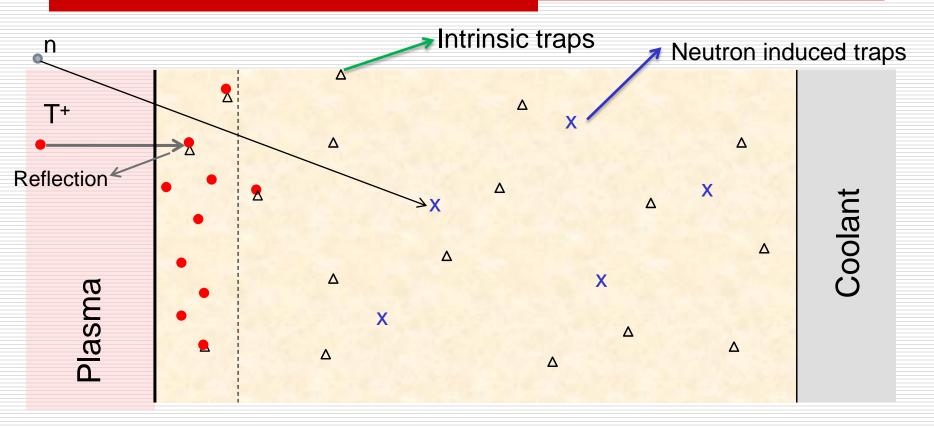
Outline

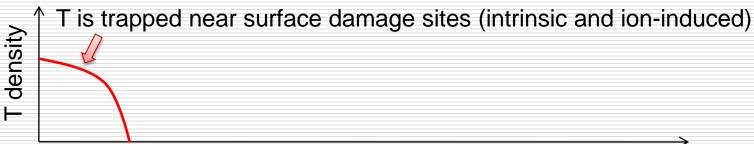
- Background and Purpose
 - General explanation of tritium behavior in wall materials
- Experimental details
 - Permeation apparatus
 - W specimens
- Pure D ion driven permeation
 - Microstructure (annealing temperature) dependence
- ☐ He/D mixed ion driven permeation
- C/D mixed ion driven permeation
- Discussion and Summary

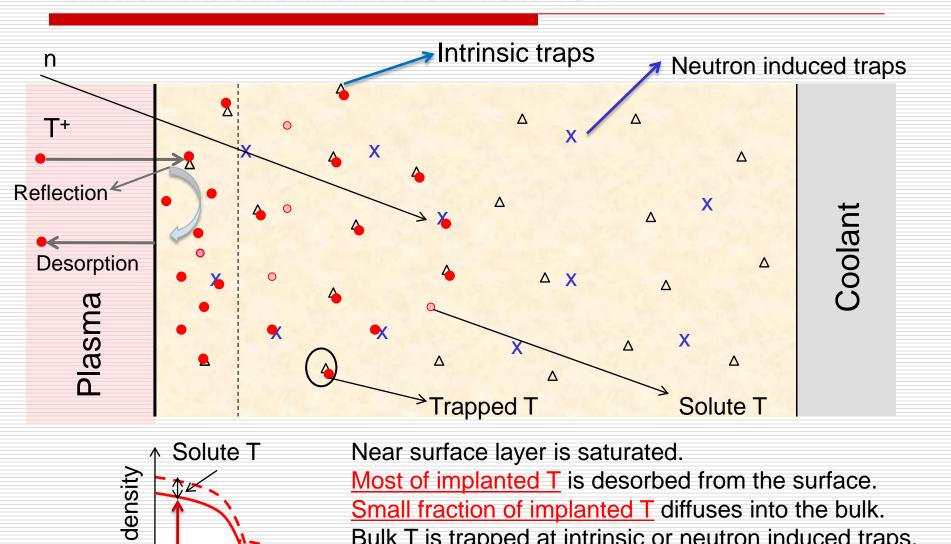
Introduction

- T retention and permeation to coolant in blankets need to be studied for evaluation of T economy and safety.
- Tungsten is a leading candidate of armor materials of blankets.
- Tungsten has low diffusivity and solubility of T, and relatively high trapping energies of defects.
- Therefore, tungsten armor on reduced activation materials of blankets (Ferritic steel, V alloy) can be permeation reducer to coolant.
- However, still basic parameters to determine permeation such as <u>effective diffusivity</u>, <u>recombination coefficient</u> etc. are not well determined.
- ☐ In addition, <u>effects of mixed ion irradiation (D+He,C)</u> are not known well either.





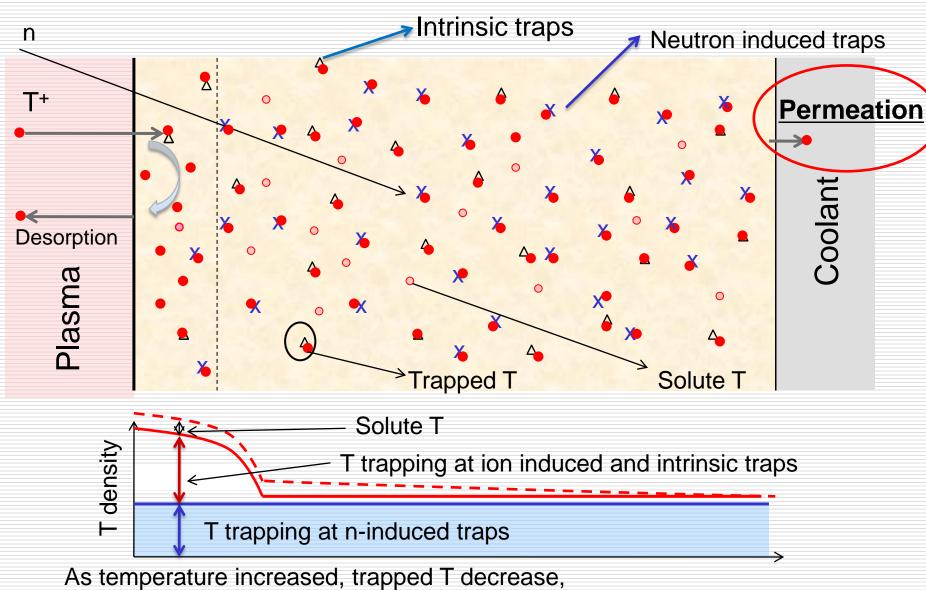




Bulk T is trapped at intrinsic or neutron induced traps.

T trapping at ion induced and intrinsic defects
T trapping at n-induced traps

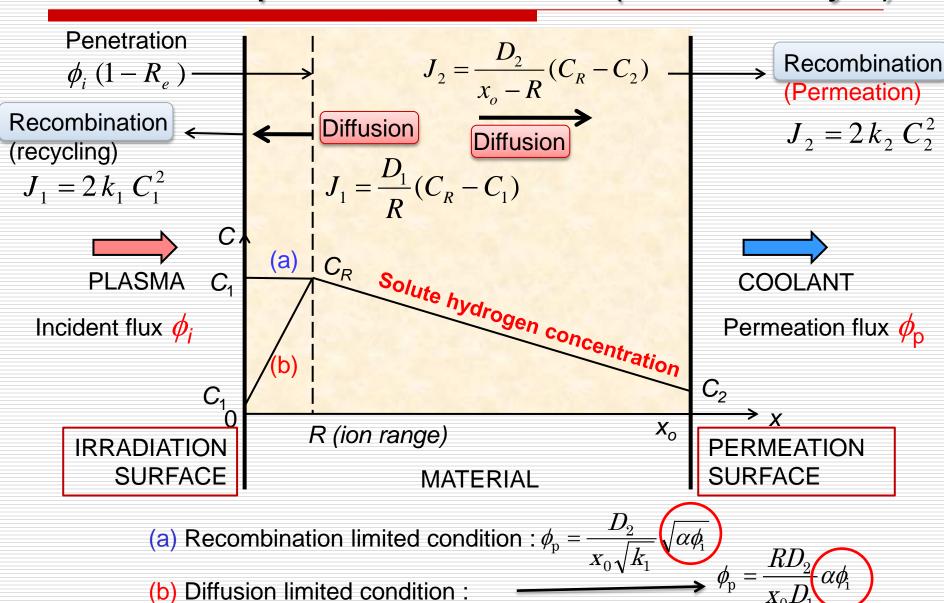
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but still traps could affect effective diffusivity

Necessary information for T behavior

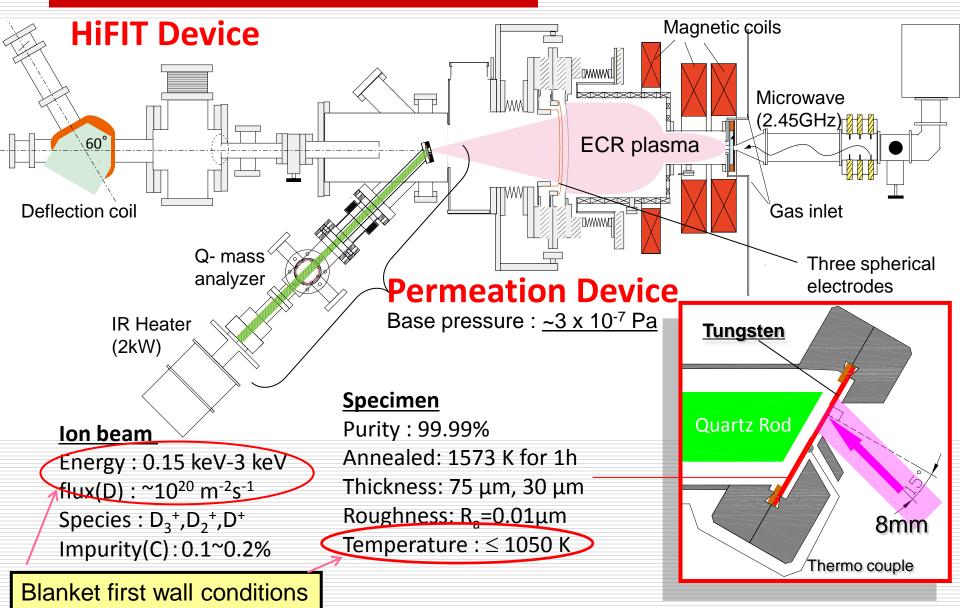
- Surface damage effects (How they affect T retention and release)
 - Recoil damage effect
 - Effects of bubbles and blisters (due to oversaturation etc.)
 - Impurity mixing effects (He, wall impurities (C, Be etc.)
- ☐ Effective diffusion mechanism (How they affect T transport)
 - Trap sites effect
 - Microstructure effect (intergranular diffusion, grain boundary diffusion)
- Neutron induced traps (How they affect T retention)
 - Trapping energies
 - Production rate (as a function of dpa)
 - Saturation level with respect to dpa



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Experimental setup

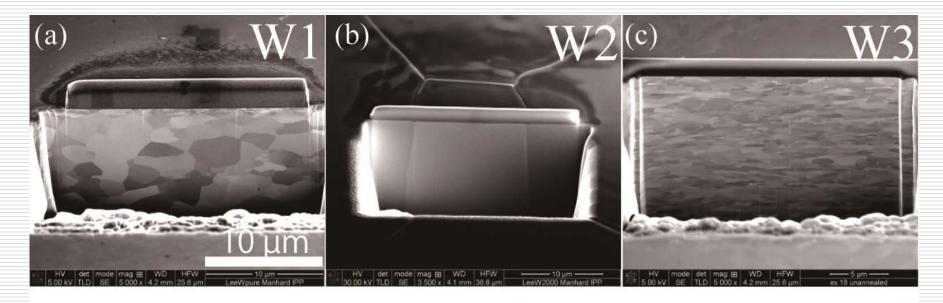
Experimental device



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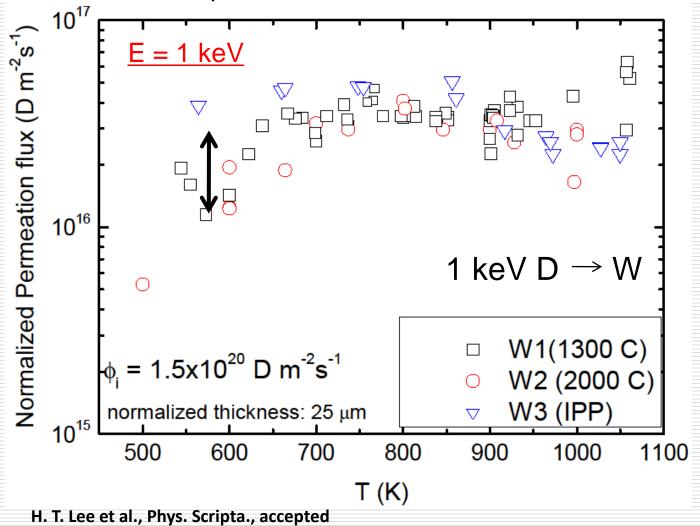
Tungsten specimens

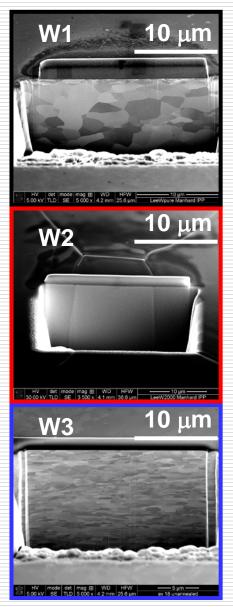
- □ W1: Sintered polycrystalline W → annealed at 1573 K.
 - Standard sample in our experiments (medium grains and low density defects)
- □ W2 : Sintered polycrystalline W → annealed at 2273 K.
 - Large grains and almost no defects
- W3 : Sintered polycrystalline W → annealed at 1203 K.



Microstructure dependence—SS permeation--

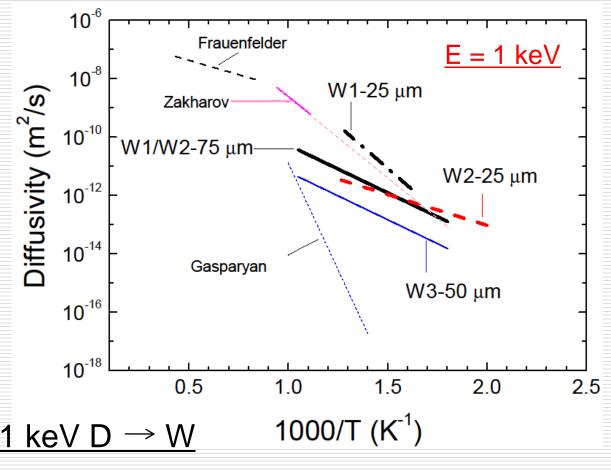
- Weak grain boundary dependence (factor of two)





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Microstructure dependence -Effective diffusivity-



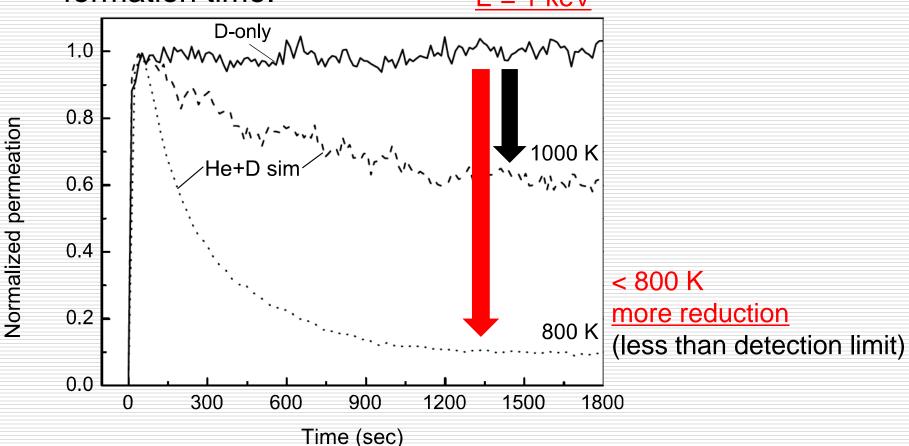
- Effective diffusivity values determined from lag time measurements.
- There are some dependences on thickness and microstructure.
- •Effective diffusivities are close to Zakharov's, but much less than Frauenfelders.
 - •This could be the effect of some traps.
- Need more investigation

R. Frauenfelder, J. Vac. Sci. Techn. 6 (1969) 388. A.P. Zakharov, E.I. Evko. Fiz. Khim. Mekh. Mater. 9 (1973) 29. Yu.M. Gasparyan et al., J. Nucl. Mater. 390 (2009) 606-609.

He/D mixed ion driven permeation

He/D mixed ion driven permeation

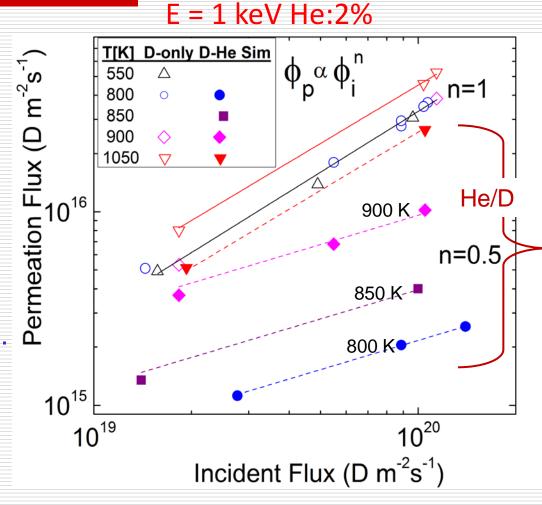
- Addition of <u>He (2%)</u> greatly reduces permeation flux.
- ☐ Saturation time almost corresponds to He bubble formation time. E = 1 keV



H. T. Lee et al., J. Nucl. Mater. (accepted)

Incident flux dependence

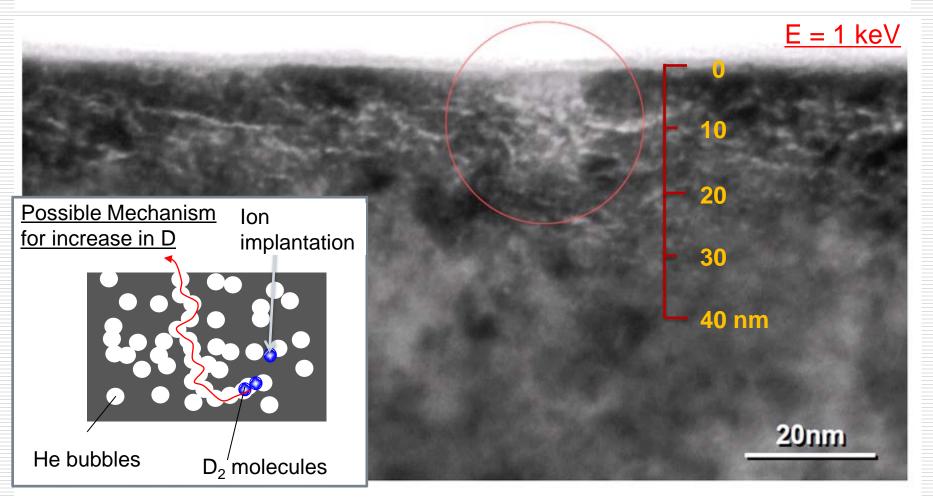
- $\square \quad \phi_{p} \sim \phi_{i} (\underline{D} \text{ only irradiation})$
- $\square \quad \phi_{p} \sim \phi_{1}^{1/2} (\underline{D/He} \text{ irradiation})$
 - $\phi_{\rm p}$: Permeation flux
 - ϕ_i : Incident flux
- ☐ Change of flux dependence suggests D release from the front surface could change from diffusion limited (D) to recombination limited (D/He).
- ☐ Two possibilities
 - Diffusion increased.
 - Recombination decreased.



Permeation flux $\phi_{\rm p}$ vs. Incident flux $\phi_{\rm i}$

Depth profile of He bubble layer

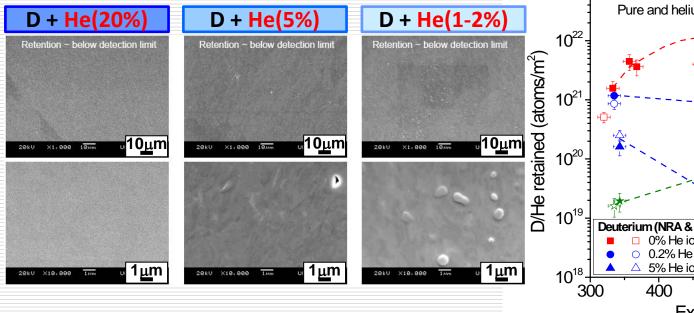
- He bubble layer was observed up to the depth of about 30 μm.
- Thickness of He bubble layer was larger than ion range (~10 μm).
- He bubbles are interconnected to form



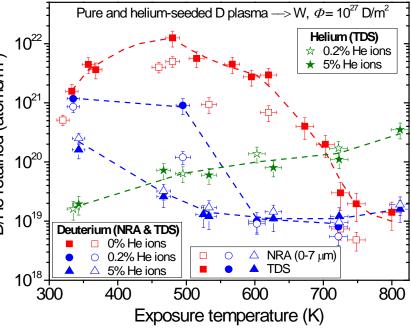
He/D effects on retention and blistering

- Blistering disappeared (or reduced) for He/D irradiation.
- D retention greatly reduced for He/D irradiation.

PISCES results (573 K)



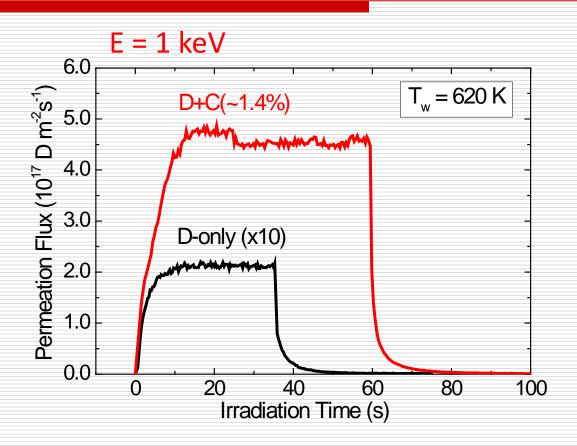
M. Miyamoto et al., Nucl. Fusion 49 (2009) 065035 (7pp).



V. Alimov et al., 12th ITPA (SOL/DIV)

C/D mixed ion driven permeation

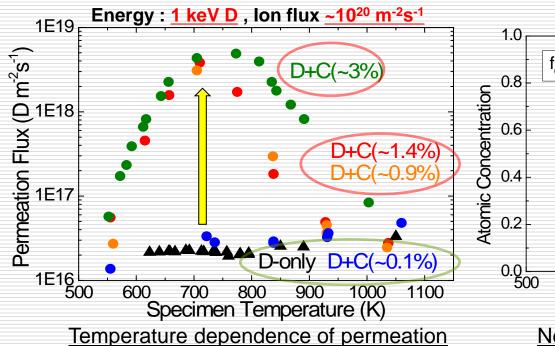
C/D mixed ion driven permeation

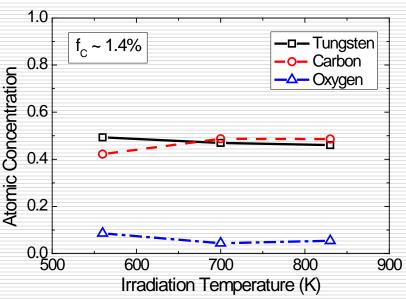


- The steady state D permeation flux for simultaneous D+C case is 20 times larger than that of D-only case.
- The lag time for simultaneous D+C case is 1.5 times larger than that of D-only case

Temperature & concentration dependence

- D permeation greatly increased even with C (>0.9%) in ion beam.
- Strong temperature dependence.
- □ Surface elemental composition shows little dependence on temperature (C:1.4%). → only temperature affect change of parameters
- ☐ Maximum increment factor in the present exp. : ~250

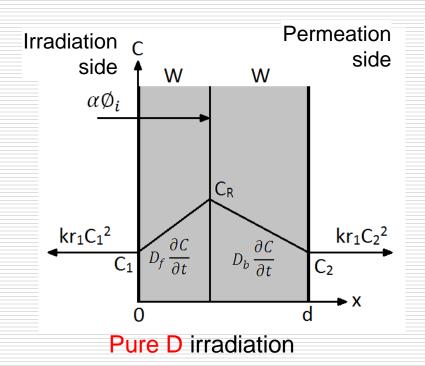


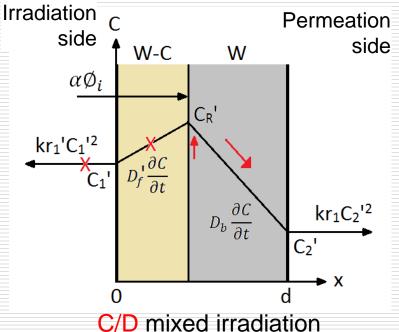


Near surface atomic composition

How C/W mixed layer affect permeation

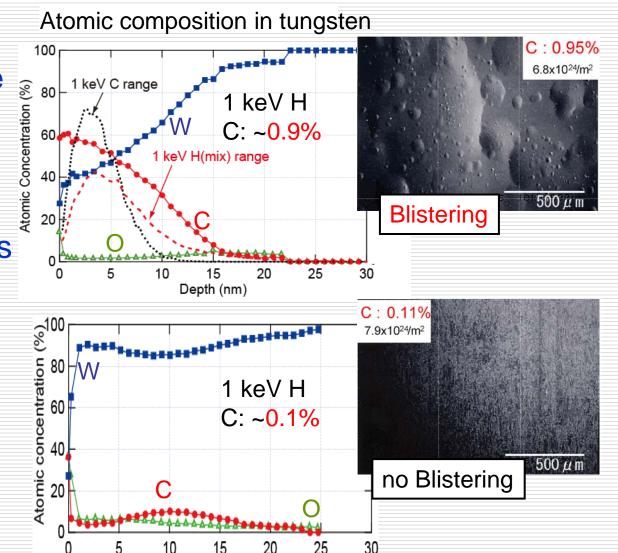
- Surface carbon concentration was determined by the balance between re-erosion and implantation of C ions.
- □ Either significant reduction in surface recombination (more than 10⁻⁴) or reduction in diffusion (10⁻²) could cause significant increase in permeation.





Effect of C/W mixed layer on blistering

- For high carbon concentration in the beam (C:0.9%), blister appeared.
- □ C/W mixed layer effectively increases T diffusion into the bulk over certain fluence.



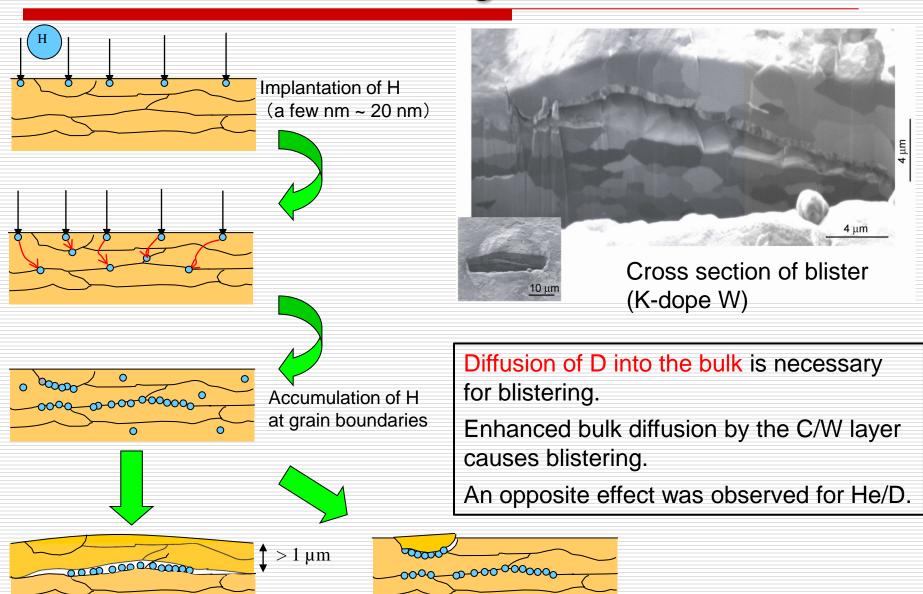
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Depth (nm)

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Mechanism for blistering

Dome-like blisters



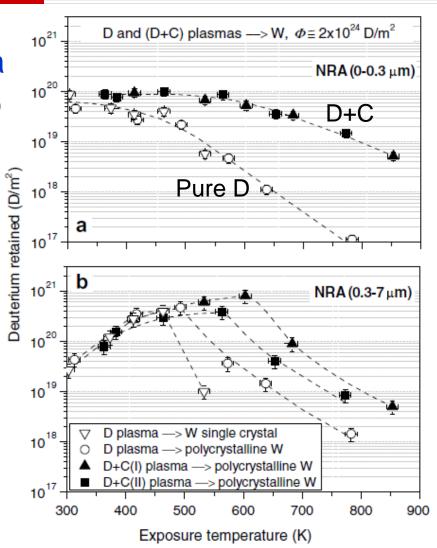
grain ejection

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D/C mixed plasma exposure raised retention

- Planar DC magnetron plasma
 - Energy : ~200 eV (D₂+ mainly)
 - Flux : 1 x 10²¹ m⁻²s⁻¹
 - C plate on cathode surface to provides C into plasma
- □ For D+C, D retention near surface (a) and bulk (b) increased at elevated temp.

V. Alimov et al., J. Nucl. Mater. 375 (2008) 192.

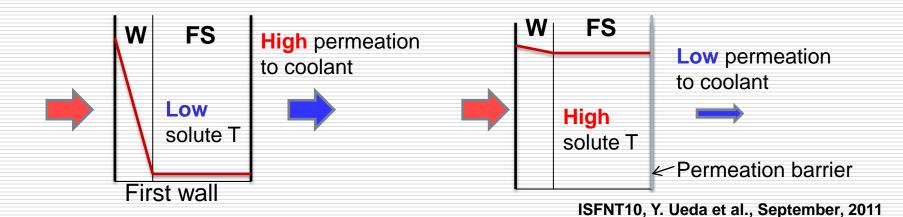


D retention in W exposed to pure D plasma ($\Box \nabla$) and D+C plasma ($\blacktriangle \blacksquare$)

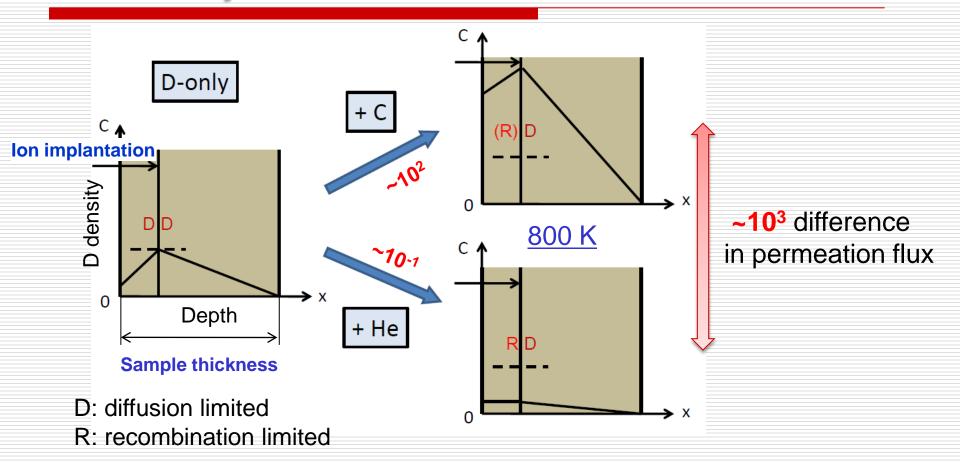
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Comments on T behavior in blankets

- Surface phenomena greatly affect whole T behavior in blankets.
- Permeation barrier increases T concentration in materials
 - To reduce T permeation by putting permeation barrier on the permeation side, solute T density increase. → Does it affect material performance?
 - □ 10⁻⁵ D/W for C/D irradiation (1.4%C, 800 K) (Assumption: Zakharov's effective diffusivity)
 - Trapped T density (equilibrium with solute T) increases with the increase in solute T.



Summary on D behavior for mixed irradiation



- Mechanism of permeation reduction (He/D) and enhancement (C/D)
 - **Addition** of He→ Effective diffusion near surface area increased.

Probably due to dense He bubble structure

Addition of C→ Recombination or diffusion reduced : under investigation

Concluding summary

- Mixed ion irradiation (He/D and C/D) into tungsten significantly changes permeation.
 - Addition of He: decrease (10⁻¹ at 800 K, and more at <800 K)</p>
 - ☐ Correlation with blister suppression
 - □ Correlation with decrease in retention
 - Addition of C: increase by two orders
 - Correlation with blister enhancement
 - ☐ Correlation with increase in retention
 - Strong temperature dependence
 - He or C effect significant at ~800 K
 - → close to blanket surface temperature (FS based)
 - Indicating importance to include material mixing effects in T modeling in wall materials
- Microstructure dependence on permeation
 - No significant effects of microstructure (W1, W2, W3) on steady-state permeation

Preferable for 1st wall